

Conference Paper

Effect of Stress on Irradiation-induced Creep and Swelling of Fe-18Cr-10Ni-Ti Steel Pressurized Specimens Irradiated in the BOR-60 Reactor

Evgenii Makarov, Victor Neustroev, Sergey Belozerov, Dmitrii Markelov, and Aleksandr Obukhov

JSC "State Scientific Center - Research Institute of Atomic Reactors", Dimitrovgrad, Russia

Abstract

The paper presents the data on the effect of stress of various signs on the irradiationinduced creep strain and swelling of austenitic Fe-18Cr-10Ni-Ti steel pressurized specimens. The pressurized specimens of standard and contoured geometry were irradiated in the BOR-60 reactor up to the damage dose of 90 and 36 dpa, accordingly, under various stress levels applied. Presented are the data resulted from TEM investigations of pressurized specimens performed with the use of the transmission electron microscope.

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Corresponding Author:

Victor Neustroev

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1. INTRODUCTION

The work to justify both the possible service-life extension of the internals from operated VVER-1000 reactors and performance of internals materials from new VVER reactors up to 60 years is rather relevant to the present day. Areas with tensile and compression stresses can appear in the contoured internals with a large number of orifices for cooling under the temperature gradients. It is therefore important to investigate the effect of stress on the material properties and structure. To perform the investigations, the austenitic Fe-18Cr-10Ni-Ti steel was chosen as the internals material of VVER reactors under both operation and development.

The work to investigate the effect of tensile stress on the material properties and structure has already been performed in RIAR, Russia and abroad, but the effect of compression stress has been studied negligibly [1-5]. Several articles devoted to the problem [5-7] exist, but the results provided need additional confirmation.

Therefore, the purpose of this work is to experimentally determine the effect of stress on the irradiation-induced swelling and creep strain in Fe-18Cr-10Ni-Ti austenitic



steels as well as on the steel microstructure under neutron irradiation in the BOR-60 fast reactor.

2. MATERIALS AND EXPERIMENTAL METHODS

The pressurized contoured specimens made of Fe-18Cr-10Ni-Ti steel were irradiated in the BOR-60 reactor up to a damage dose of 36 dpa and standard specimens were irradiated up to 90 dpa at a temperature not exceeding 350°C [8]. Compression and tensile stresses were simultaneously applied to the claddings of irradiated specimens of the contoured geometry. No-stress specimens were irradiated at the same time as the above ones. In this study, we used the TEM methods to investigate the structure of the material and measure linear dimensions of the pressurized specimens in order to determine the irradiation-induced creep strain and swelling. TEM investigations of the steel microstructure were done using the co-axial pressurized specimens irradiated in the BOR-60 reactor up to a damage dose of 36 dpa at temperatures of 340 and 420 °C.

3. RESULTS AND DISCUSSION

3.1. Investigations results related to austenitic o8X18H1oT steel standard pressurized specimens irradiated at a temperature of 340 °C up to a damage dose of ~ 90 dpa

Based on the performed investigations and analysis results, the mechanisms of changes in the length and diameter of the pressurized specimens were verified. It is shown (Fig. 1) that the length and diameter of the pressurized specimens grow proportionally with the damage dose growth under various levels of stress applied. The dependence between the relative diameter increment in the Fe-18Cr-10Ni-Ti steel pressurized specimens and the stress and damage dose is close to linear.



Figure 1: Dose-length (a) and dose-diameter (b) dependencies in the standard Fe-18Cr-10Ni-Ti steel pressurized specimens irradiated in the BOR-60 reactor at 340°C, at various stress levels.



The creep modulus calculation [9] shows that for the standard specimens with stresses 150-190 MPa irradiated in the BOR-60 reactor, it tends to a certain level of saturation of about $2.0 \cdot 10^{-6}$ (MPa \cdot dpa)⁻¹ at a damage dose higher than 40 dpa and irradiation temperature of 340°C.

3.2. Investigations results related to austenitic Fe-18Cr-10Ni-Ti steel co-axial pressurized specimens irradiated at a temperature of 420°C up to a damage dose of ~ 36 dpa

The specimens were located in one of the levels of the BOR-60 irradiation rig suspension. During the reactor outages, the diameter and length of the specimens were measured. The measurements were done in the BOR-60 hot cell using contact methods having the length measurement error of \pm 10 μ m and diameter measurement error of \pm 5 μ m.

The investigations and further analysis of the results proved the earlier obtained dose dependencies for changes in the length and diameter of the pressurized specimens. It was shown that as the damage dose became higher, the length and diameter of the pressurized specimens grew as well under different stresses applied. The dependence between the increment in the diameter of the Fe-18Cr-10Ni-Ti steel contoured specimens and the damage dose was close to linear.

The creep modulus was calculated to be equal to $2.0 \cdot 10^{-6}$ (MPa \cdot dpa)⁻¹ at T_{irr.} = 340°C that corresponds to the creep modules of other austenitic steels at the said temperatures.

3.3. Investigations results related to austenitic Fe-18Cr-10Ni-Ti steel co-axial pressurized specimens irradiated at a temperature of 340°C up to a damage dose of \sim 36 dpa

The pressurized contoured specimens made of Fe-18Cr-10Ni-Ti steel were irradiated in the BOR-60 reactor up to a damage dose of 36 dpa at \sim 420°C. Compression and tensile stresses were simultaneously applied to the claddings of irradiated specimens. No-stress specimens were irradiated at the same time as the above ones.

The experiment and analysis of the results proved the earlier obtained dose dependencies for changes in the length and diameter of the pressurized specimens. It is



shown (Fig.2) that as the damage dose became higher, the length and diameter of the pressurized specimens grew as well under different stresses applied.



Figure 2: Dose-length (a) and dose-diameter (b) dependencies in the Fe-18Cr-10Ni-Ti steel co-axial pressurized specimens irradiated in the BOR-60 reactor up to 36 dpa at 420°C.

3.4. Examination of the microstructure of irradiated Fe-18Cr-10Ni-Ti steel pressurized specimens

The TEM microscope JEM-2000FXII was used to examine the microstructure of the pressurized specimens irradiated up to 36 dpa at 420 and 340°C.

3.4.1. 420°C

TEM examination revealed the presence of similar constituents in the irradiated Fe-18Cr-10Ni-Ti steel pressurized specimens, i.e. vacancies of different diameters and concentration, dislocation loops and secondary phase particles (Fig.3). The swelling and average void diameter rise in case of both compression and tensile stresses are applied (Fig.4). However, the voids concentration in the specimens did not change practically as both applied stresses became higher (Fig.5).



Figure 3: Vacancies in the austenitic Fe-18Cr-10Ni-Ti specimens irradiated in the BOR-60 reactor up to 36dpa at 420°C: a) – no stress; b) – tensile tangential stress + 206 MPa; c) – compression tangential stress – 110 MPa.





Figure 4: Voids swelling (a) and voids average diameter (b) vs. stress in Fe-18Cr-10Ni-Ti specimens irradiated up to 36 dpa.



Figure 5: Voids concentration vs. stress in Fe-18Cr-10Ni-Ti specimens irradiated up to 36 dpa.

3.4.2. 340°C

Since both irradiation temperature and damage dose for the specimens in question were the same, then the microstructure of all specimens was also practically the same. Equiaxial grains of austenite were observed in all specimens as well as secondary phase globular precipitates distributed uniformly in the material bulk. The globular precipitates are particles of titanium carbonitride Ti(C, N) and can be found in this steel even in the initial state. The size of particles ranged from 0.1 μ m to 1.2 μ m; their maximal concentration made up about $3 \times 10^{17} \text{m}^{-3}$. These particles can be found both in the grain body and at the inter-grain boundaries.

Single α -Fe phase precipitates were found in a – 62 MPa specimen. Probably, this phase could be found in other specimens but it was not detected by the TEM because of





its low concentration. No voids were found in the specimens. The dislocation structure is characterized by the presence of dislocation loops, in particular Frank loops (Fig.7).





Figure 6: Dark-field images of the α -Fe phase precipitates in the – 62 MPa specimen at the grain triboundary (a) and at the edge of a crack (b) appeared during the electrolytical thinning of the specimen.



Figure 7: Dislocation structure of specimens under investigation: (a) light-field image of dislocation loops in a + 82 MPa specimen; (b) dark-field image of the Frank loops in reflex [111] in a – 41 MPa specimen, (c) dark-field image of the Frank loops in reflex [111] band in + 247 MPa specimens.

It is clearly seen that as the stress modulus rises, the loop concentration in steel increases as well (Fig.8a). No dependence was reveled between the dislocation loop size and applied stress (Fig.8b).



Figure 8: Loop concentration (a) and average loop size (b) vs. stress in Fe-18Cr-10Ni-Ti steel specimens irradiated up to 36 dpa at T_{irr} . = 340°C.



4. CONCLUSION

The investigation of austenitic Fe-18Cr-10Ni-Ti steel standard and co-axial pressurized specimens irradiated in the BOR-60 reactor showed the following:

- 1. The diameter and length of both types of pressurized specimens increase linearly as the damage dose rises;
- 2. The creep modules calculated for steel Fe-18Cr-10Ni-Ti are in good correlation with the creep modules of 18Cr-9Ni-based austenitic steels;
- TEM examinations showed a presence of vacancies in the corrosion-resistant austenitic Fe-18Cr-10Ni-Ti pressurized specimens irradiated up to 36 dpa at 420°C. As both types of stresses applied rise, the swelling and average voids diameter change slightly;
- No voids were reveled in the pressurized specimens irradiated up to 36 dpa at 340°C;
- 5. Globular particles of titanium carbonitride as well as G-phase particles were revealed. Single α -Fe phase precipitates were found in some specimens;
- 6. Dislocation Frank loops were revealed having an average diameter of about 9nm and concentration of up to $6.6 \cdot 10^{22} \text{ m}^{-3}$.
- 7. No significant difference was revealed in the effect of both tensile and compression stresses on the microstructure of steel Fe-18Cr-10Ni-Ti specimens irradiated in the BOR-60 at 340°C;
- 8. The investigation results can be used to justify the prolongation of the VVER-1000 baffle lifetime and for new VVER-1200 projects as well.

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